Oral Presentations

September 29 (Mon.) 13:00-15:30

1-1. Reactor Analysis Method (Mizuho_A)

13:00	TBD	Hideo Soneda	Hitachi-GE Nuclear Energy, Ltd.	2000002
13:20	Development of Enhanced SPH Method for Pin-by-Pin Core	Satoshi Takeda	NUCLEAR FUEL INDUSTRIES, LTD	1103613
13:40	STUDY ON CROSS SECTION CORRECTION USING SPH METHOD FOR A WHOLE CORE MOC CALCULATION	Akinori Giho	Shikoku Electric Power Co., Inc.	1084323
14:00	ASYMPTOTIC, MULTIGROUP FLUX RECONSTRUCTION AND CONSISTENT DISCONTINUITY FACTORS	Travis John Trahan	Los Alamos National Laboratory	1106814
14:20	A POSTERIORI RECONSTRUCTION OF THE FLUX PROFILE IN THE CASE OF LOCALIZED AXIAL HETEROGENEITIES: AN APPLICATION TO THE MODELING OF PWR MIXING GRIDS	Enrico Girardi	Electricite de France - R&D	1101880
14:40	Effects of Advanced Radial Submeshing Methods on Pin Power Reconstruction for an EPR Core Design	Petra Mala	Paul Scherrer Institute	1105669
15:00	STUDY ON ROBUST ENERGY GROUP STRUCTURE TO SPECTRAL INTERFERENCE FOR PWR PIN-BY-PIN CORE ANALYSIS	Satoshi Wada	Osaka university	1105989

SS2-1. Reactor Physics and Criticality Safety Activities in OECD/NEA Working Party (Mizuho_B)

13:00	Activities of OECD/NEA on Scientific Issues of Reactor Systems and Critical Safety - Current Status and Future Plan	TBD	OECD/NEA	2000000
13:20	UNCERTAINTY ANALYSIS OF THE OECD/NRC OSKARSHAMN-2 BWR STABILITY BENCHMARK	Ivan Angelov Gajev	Royal Institute of Technology	1093239
13:40	ANALYSIS OF THE OECD/NEA OSKARSHAMN-2 FEEDWATER TRANSIENT AND STABILITY BENCHMARK WITH SIMULATE-3K	Abdelhamid Dokhane	Paul Scherrer Institute (PSI)	1101708
14:00	Data Assimilation for Kinetic Parameters Uncertainty Analysis	Evgeny Ivanov	Institut de Radioprotection et Surete	1105973
14:20	Criticality and Reactor Physics Benchmark Experiments: Influence of Nuclear Data Uncertainties	Winfried Zwermann	Gesellschaft fuer Anlagen- und Reaktorsicherheit (GRS) mbH	1103852
14:40	THE EVALUATION OF THE SUBCRITICAL EXPERIMENTS PERFORMED IN THE IPEN/MB-01 RESEARCH REACTOR FACILITY FOR THE IRPhE PROJECT	Adimir dos Santos	Instituto de Pesquisas Energeticas e Nucleares IPEN-CNEN/SP	1095691
15:00	POLARIS: A NEW TWO-DIMENSIONAL LATTICE PHYSICS ANALYSIS CAPABILITY FOR THE SCALE CODE SYSTEM	Matthew A Jessee	Oak Ridge National Laboratory	1127872

SS1-1. Molten Salt Reactors (Hiei)

13:00	EXPERIMENTAL MODELLING AND NUMERICAL ANALYSIS OF A MOLTEN SALT FAST REACTOR	Bogdán Keisuke Yamaji	Institute of Nuclear Techniques, Budapest University of Technology and Economics	1106911
13:20	Remark on the propagating neutron noise in a MSR	Victor Dykin	Chalmers University of Technology	1127633
13:40	The two-group point-kinetic component of neutron noise in an MSR	Victor Dykin	Chalmers University of Technology	1105994
14:00	NEUTRONICS OF FLUID FUEL SYSTEM WITH PERFECT REMIXING	Sandra Dulla	Politecnico di Torino	1105851
14:20	An innovative approach to dynamics modeling and simulation of the Molten Salt Reactor Experiment	Matteo Zanetti	Politecnico di Milano, Department of Energy, Nuclear Engineering Division	1104090
14:40	SAFETY CRITERIA AND GUIDELINES FOR MSR ACCIDENT	Ritsuo Yoshioka	International Thorium Molten-Salt Forum	1081443
15:00	Reactivity-Insertion-Accident Analysis without Scram of Molten Salt	Yang Yang	Shanghai Institute of Applied Physics	1087405

SS5. Multiscale, Multiphysics Approaches in Nuclear Science and Engineering Applications (Atago)

13:00	Influence of an SN solver in a fine-mesh neutronics/thermal-hydraulics framework	Klas Jareteg	Division of Nuclear Engineering, Department of Applied Physics, Chalmers University of Technology	1104064
13:20	High-Fidelity Multi-Physics Calculations for Light Water Reactors Using Coupled CTF/TORT-TD/FRAPTRAN	Jeffrey William Magedanz	The Pennsylvania State University	1106221
13:40	THE COUPLING OF the NEUTRONIC TRANSPORT APPLICATION RATTLESNAKE TO THE NUCLEAR FUELS PERFORMANCE APPLICATION BISON UNDER THE MOOSE FRAMEWORK	Frederick N Gleicher	Idaho National Laboratory	1106401
14:00	A model of two-stage core calculation method coupled with subchannel analysis for boiling water reactors	Takeshi Mitsuyasu	Hitachi, Ltd.	1100830
14:20	SUBSPACE METHODS FOR MULTI-PHYSICS REDUCED ORDER MODELING IN NUCLEAR ENGINEERING APPLICATIONS	Bassam A. Khuwaileh	Department of Nuclear Engineering North Carolina State University, Raleigh, NC,	1126241
14:40	EFFICIENT FINITE ELEMENT FIELD INTERPOLATION FOR MULTIPHYSICS APPLICATIONS	Jean C Ragusa	Texas A&M University	1106423
15:00	Extension of the Entropy Viscosity Method to Flows with Friction Forces and Source Terms	Jean C Ragusa	Texas A&M University	1106855

11-1. Research Reactors and Spallation Sources (Cosmos)

13:00	An updated core design for the multi-purpose irradiation facility	Gert Van den Eynde	SCK-CEN	1104788
13:20	FEASIBILITY STUDY OF INSTALLING A THERMAL TO 14 MeV FEASIBILITY STUDY OF INSTALLING A THERMAL TO 14 MeV	Luka Snoj	Jozef Stefan Institute	1102618
13:40	Development and Validation of a New APOLLO2-Based Calculation Scheme Dedicated to Ex-Core Rod Irradiations in the OSIRIS MTR	Florent Chevallier	Alternative Energies and Atomic Energy Commission (CEA)	1106200
14:00	NEUTRONIC DESIGNS AND ANALYSES OF A NEW CORE-MODERATOR ASSEMBLY AND NEUTRON BEAM PORTS FOR THE PENN STATE BREAZEALE REACTOR	Dundar Ucar	The Pennsylvania State University	1084741
14:20	Design Studies for a Multiple Application Thermal Reactor for Irradiation experiments (MATRIX)	Michael A Pope	Idaho National Laboratory	1085708
14:40	SIMULATED IRRADIATION OF SAMPLES IN HFIR FOR USE AS POSSIBLE TEST MATERIALS IN THE MPEX (MATERIAL-PLASMA EXPERIMENTAL FACILITY)	Ronald J. Ellis	Oak Ridge National Laboratory	1178823

15:00	MCNPX ANALYSIS OF DELAYED NEUTRON FRACTION IN BERYLLIUM REFLECTED CORES	SILVA KALCHEVA	SCK-CEN	1077740

September 29 (Mon.) 15:45-18:15

1-2. Reactor Analysis Method (Mizuho_A)

15:45	APPLICATION OF THE EFFICIENT CONSISTENT SPATIAL HOMOG-ENIZATION METHOD IN NEUTRON TRANSPORT THEORY TO A GAS COOLED THERMAL REACTOR PROBLEM	Saam Yasseri	Georgia Institute of Technology	1106236
16:05	APPLICATION OF THE HYBRID DIFFUSION-TRANSPORT SPATIAL HOMOGENIZATION METHOD TO A HIGH TEMPERATURE TEST REACTOR BENCHMARK PROBLEM	Gabriel Kooreman	Georgia Institute of Technology	1106275
16:25	NORMALIZATION METHODS FOR DIFFUSION CALCULATIONS WITH VARIOUS ASSEMBLY HOMOGENIZATIONS	Coline Brosselard	EDF R&D/SINETICS	1094770
16:45	ON THE PRACTICAL FEASIBILITY OF CONTINUOUS-ENERGY MONTE CARLO IN SPATIAL HOMOGENIZATION	Jaakko Leppanen	VTT Technical Research Centre of Finland	1103983
17:05	A Dynamic Homogenization Model for Pebble Bed Reactors	Maurice Grimod	CEA de Saclay	1105216
17:25	HOMOGENIZATION OF THE STEP CHARACTERISTIC SCHEME IN PHASE SPACE	Dmitriy Anistratov	North Carolina State University	1126229
17:45	Spatial Rehomogenization of Cross Sections and Discontinuity Factors for Nodal Calculations	Aldo Dall'Osso	AREVA NP	1086844

SS2-2. Reactor Physics and Criticality Safety Activities in OECD/NEA Working Party (Mizuho_B)

15:45	EVALUATION OF LARGE 3600MWth SODIUM-COOLED FAST REACTOR OECD NEUTRONIC BENCHMARKS	Laurent BUIRON	CEA	1103961
16:05	Evaluation of Medium 1000 MWth Sodium-cooled Fast Reactor OECD neutronic Benchmarks	Nicolas Emile Stauff	Argonne National Laboratory	1104111
16:25	SFR WHOLE CORE BURNUP CALCULATIONS WITH TRIPOLI-4 MONTE CARLO CODE	Yi-Kang Lee	CEA-Saclay	1106242
16:45	SUMMARY AND STATUS OF OECD/NEA UAM-LWR BENCHMARK	Maria Nikolova Avramova	The Pennsylvana State University	1104813
17:05	UNCERTAINTY AND SENSITIVITY ANALYSIS OF OECD/NEA UAM FUEL THERMAL BEHAVIOUR BENCHMARK USING A FALCON/URANIE METHODOLOGY	Younsuk Yun	Paul Scherrer Institut	1105847
17:25	New PSI methodology for manufacturing and technological uncertainty quantification	Marco Pecchia	Paul Scherrer Institut (PSI)	1105694
17:45	RE-EVALUATION AND CONTINUED DEVELOPMENT OF SHIELDING BENCHMARK DATABASE SINBAD	Ivan Alexander Kodeli	Jozef Stefan Institute	1106058

SS1-2. Molten Salt Reactors (Hiei)

15:45	HYBRID SPECTRUM MOLTEN SALT REACTOR	Jiri Krepel	PSI Switzerland	1102672
16:05	Thorium Conversion Optimization in Two-Fluid Molten-Salt Reactor	Jan Frybort	UJV Rez / Czech Technical University in Prague	1105569
16:25	Development of Computer Code Systems for Molten Salt Reactor Core Analysis	Yongjin Jeong	Ulsan National Institute of Science and Technology	1106010
16:45	Use of MCDancoff Factor Correction for Multi-group Fuel Depletion Analyses of Liquid Salt Cooled Reacotors	Michael Huang	Georgia Institute of Technology	1106115
17:05	COMPARATIVE STUDIES ON PLUTONIUM AND 233U UTILIZATION IN MINIFUJI MSR	Abdul Waris	Bandung Institute of Technology	1108342
17:25	ON AN OPTIMIZED NEUTRON SHIELDING FOR AN ADVANCED MOLTEN SALT FAST REACTOR DESIGN	Bruno Merk	Helmholtz-Zentrum Dresden-Rossendorf e.V.	1105793

SS7. Control Rod Withdrawal Tests Performed During the PHENIX End-of-Life Experiments (Atago)

15:45	IAEA BENCHMARK CALCULATIONS ON CONTROL ROD WITHDRAWAL TEST PERFORMED DURING PHENIX END-OF-LIFE EXPERIMENTS - BENCHMARK RESULTS AND COMPARISONS	Vincent PASCAL	CEA, DEN, DER, SPRC/LEDC	1096461
16:05	CEA CONTRIBUTION TO THE ANALYSIS OF THE CONTROL ROD WITHDRAWAL TEST PERFORMED DURING PHENIX END-OF-LIFE EXPERIMENTS (IAEA Common Research Program)	Vincent PASCAL	CEA, DEN, DER, SPRC/LEDC	1096440
16:25	IAEA Benchmark Calculations on Control Rod Withdrawal Test Performed During PHENIX End-of-Life Experiments - JAEA's	Kazuya Takano	Japan Atomic Energy Agency	1090681
16:45	BENCHMARK ANALYSIS OF PHENIX CONTROL ROD WITHDRAWAL END-OF-LIFE EXPERIMENTS	Devan Kunhiraman	Indira Gandhi Centre for Atomic Research	1102267
17:05	CALCULATION OF THE PHENIX END-OF-LIFE TEST "CONTROL ROD WITHDRAWAL" WITH THE ERANOS CODE	Evgeny Ivanov	Institut de Radioprotection et Surete Nucleaire	1104099
17:25	Simulation of PHENIX Control Rod Withdrawal Experiments with	Vladimir Kriventsev	Karlsruhe Institute of Technology (KIT)	1104108
17:45	STUDY OF THE EFFECT OF HETEROGENEITY OF THE CONTROL RODS IN THE PHENIX REACTOR	FABRIZIO GABRIELLI	Karlsruhe Institute of Technology (KIT), Institute for Nuclear and Energy Technologies (IKET)	1106079

11-2. Research Reactors and Spallation Sources (Cosmos)

15:45	A method for reactivity monitoring in subcritical source-driven systems	Sandra Dulla	POLITECNICO DI TORINO	1076430
16:05	INTEPRETATION OF EXPERIMENTAL MEASUREMENTS ON THE SC-1 CONFIGURATION OF THE VENUS-F CORE	Sandra Dulla	Politecnico di Torino	1107269
16:25	Neutronic Characteristics of Solid Targets in Accelerator-Driven System at Kyoto University Critical Assembly	Cheolho Pyeon	Kyoto University	1111120
16:45	EVALUATION OF NEUTRON SPECTRUM AT IN-CORE IRRADIA-TION EQUIPMENTS IN KUR WITH LOW ENRICHED URANIUM FUEL	Tadafumi Sano	Kyoto University Research Reactor Institute	1126220
17:05	ACCULACY OF THORIUM-LOADED ACCELERATOR-DRIVEN SYSTEM EXPRIMENTS AT KYOTO UNIVERSITY CRITICAL	Masao Yamanaka	Graduate school of Energy Science, Kyoto University	1090583

17:25	SHUTDOWN TRANSIENTS ANALYSIS FOR REFLECTOR DEVICES POWER CALCULATIONS IN JULES HOROWITZ MATERIAL TESTING REACTOR (JHR)	Patrizio Console Camprini	ENEA	1104755
17:45	Neutronic Analysis of the PULSTAR Reactor Using Monte Carlo	Ayman I. Hawari	North Carolina State University	1127882

September 30 (Tue.) 8:00-10:05

1-3. Reactor Analysis Method (Mizuho_A)

8:00	Research Reactor In-Core Fuel Management Optimisation Using The Multiobjective Cross-Entropy Method	Evert B. Schlunz	South African Nuclear Energy Corporation SOC Ltd (Necsa)	1101850
8:20	Transient Cycle Fuel Management Optimization of a Pressurized Water Reactor	tongkyu park	FNC Technology Co., Ltd.	1105495
8:40	A MULTI-LEVEL PARALLEL COMPUTATION OF REACTOR CORES USING GPU FOR LOADING PATTERN OPTIMIZATION	Takuya Okubo	Nagoya University	1101184
9:00	Exact-to-Precision Generalized Perturbation Theory for Reactor Design Caculations	Congjian Wang	North Carolina State University	1127680
9:20	Depletion GPT-Free Sensitivity Analysis of the TMI Reactor Eigenvalue	Christopher Brandon Kennedy	North Carolina State University	1106622
9:40	The "Virtual Density" Theory of Neutronics: A Generic Method for Geometry Distortion Reactivity Coefficients	Mark Reed	TerraPower	1106665

5-1. Nuclear Criticality Safety (Mizuho_B)

8:00	NUCLEAR CRITICALITY SAFETY IN THE UNITED STATES: RECENT EVENTS, TRENDS AND A REVIEW OF THE SAFETY CULTURE	Matthew S Hodges	University of Nevada, Las Vegas (UNLV)	1105240
8:20	A NEW OECD/NEA DATABASE OF NUCLIDE COMPOSITIONS OF SPENT NUCLEAR FUEL	Franco Michel-Sendis	OECD Nuclear Energy Agency	1123747
8:40	OECD EGBUC Benchmark VIII ? Comparison of calculation codes and methods for the analysis of small-sample reactivity experiments	Pierre LECONTE	CEA Cadarache	1104030
9:00	Criticality Calculation of Fuel Debris in Fukushima Daiichi Nuclear Power Station	Akiyuki Tsuchiya	Hitachi-GE Nuclear Energy, Ltd.	1107488
9:20	Design of an efficient calculation model of BWR cold critical experiments for validation	Anssu Ranta-aho	Teollisuuden Voima Oyj	1106047
9:40	FIRST BURNUP CREDIT APPLICATION FOR TRANSPORT AND STORAGE CASK USING FRENCH EXPERIMENTS	Marcel TARDY	AREVA	1106235

2-1. Deterministic Transport Theory (Hiei)

8:00	A Collision Probability Based Method to Compute Cross Sections Sensitivities for the Subgroup Self-Shielding Technique	Maxime Dion	Ecole Polytechnique de Montreal	1080046
8:20	Improvement of a convergence technique for MOC calculation with large negative self-scattering cross section	Masato Tabuchi	Nuclear Engineering, Ltd.	1105664
8:40	Boundary Acceleration Techniques for CMFD-Accelerated 2D-MOC	Shane Gray Stimpson	University of Michigan	1105191
9:00	A Low Order Nonlinear Transport Acceleration Scheme for the Method of Characteristics	Lulu Li	Massachusetts Institute of Technology	1105319
9:20	p-CMFD ACCELERATION AND NONOVERLAPPING LOCAL/GLOBAL ITERATIVE TRANSPORT METHODS WITH 2-D/1-D FUSION KERNEL	Seungsu Yuk	Korea Advanced Institute of Science and Technology	1126428
9:40	APPLICATION OF THE SDD-CMFD ACCELERATION TECHNIQUE TO PARALLEL 3-D METHOD OF CHARACTERISTICS TRANSPORT	Brendan M. Kochunas	University of Michigan	1104795

3-1. Monte Carlo Methods (Atago)

8:00	UNSTRUCTURED MESH BASED MULTI-PHYSICS INTERFACE FOR CFD CODE COUPLING IN THE SERPENT 2 MONTE CARLO CODE	Jaakko Leppanen	VTT Technical Research Centre of Finland	1103981
8:20	ANALYZING THE STATISTICS OF GROUP CONSTANTS GENERATED BY SERPENT 2 MONTE CARLO CODE	Toni Kaltiaisenaho	VTT Technical Research Centre of Finland	1106126
8:40	Theoretical Prediction on Underestimation of Statistical Uncertainty for Fission Rate Tally in Monte Carlo Calculation	Tomohiro Endo	Nagoya University	1126624
9:00	ANALYSIS OF TALLY CORRELATION IN LARGE LIGHT WATER REACTORS	Bryan R. Herman	Massachusetts Institute of Technology	1094801
9:20	Higher-Mode Applications of Fission Matrix Capability for MCNP	Sean E Carney	University of Michigan	1105312
9:40	A Symmetric View Hiding the Ugly Truth	Dennis Mennerdahl	E Mennerdahl Systems	1106471

SS8. Reactor Physics of Non-Traditional LWR Fuel Design (Cosmos)

8:00	I2S-LWR EQUILIBRIUM CYCLE CORE ANALYSIS	Fausto Franceschini	westinghouse	1126928
8:20	Uranium nitride composite fuels in a pressurized water reactor: exploration of multi-batch cycle length and UB4 admixture for reactivity	Nicholas R Brown	Brookhaven National Laboratory	1104783
8:40	Impact of Coating on Nitride Fuel Performance in PWRs	Florent Heidet	Argonne National Laboratory	1128293
9:00	OPTIMIZATION OF FULLY CERAMIC MICRO-ENCAPSULATED FUEL ASSEMBLY FOR PWR	Massimiliano Fratoni	University of California, Berkeley	1105391
9:20	Fully Ceramic Microencapsulated Fuels: Characteristics and Potential LWR Applications	Jeffrey J Powers	Oak Ridge National Laboratory	1127750
9:40	NEUTRONIC CHALLENGES OF ADVANCED BOILING WATER REACTOR DESIGNS	Koroush Shirvan	MIT	1126323

September 30 (Tue.) 10:20-12:00

1-4. Reactor Analysis Method (Mizuho_A)

10:20	Watts Bar Unit 1 Cycle 1 Zero Power Physics Tests Analysis with	Jess C. Gehin	Oak Ridge National Laboratory	1127867
10:40	AP1000R PWR REACTOR PHYSICS ANALYSIS WITH VERA-CS AND KENO - PART I: ZERO POWER PHYSICS TESTS	Fausto Franceschini	westinghouse	1126938
11:00	AP1000 PWR REACTOR PHYSICS ANALYSIS WITH VERA-CS AND KENO - PART II: POWER DISTRIBUTION	fausto franceschini	westinghouse	1126207

11:20	SOLUTION OF THE BEAVRS BENCHMARK USING THE nTRACER DIRECT WHOLE CORE TRANSPORT CODE	Min Ryu	Seoul National University	1104549
11:40	CALCULATION OF THE PRESSURE VESSEL FLUENCE IN THE HUNGARIAN VVER-440 PLANTS FOR THE LIFETIME EXTENSION	Gyorgy Hegyi	Magyar Tudomanyos Akademia Energiatudomanyi Kutatokozpont	1084093

5-2. Nuclear Criticality Safety (Mizuho_B)

10:20	UNCERTAINTY EVALUATION OF REACTIVITY IN SINGLE AND MULTI-REGION TSUNAMI MODELING ANALYSIS FOR DRY CASK	Quentin T Newell	University of Nevada, Las Vegas (UNLV)	1087608
10:40	FAVORABLE FEATURES IN KINETICS OF FAST REACTORS WITH PHYSICALLY THICK 208Pb-REFLECTOR	Gennady Genrikhovich Kulikov	National Research Nuclear University Moscow Engineering Physics Institute	1092833
11:00	Transient Analysis in Super Critical Condition for Several Fuel-solution Tanks System with Different Layout	Haruka Kikuchi	Department of Nuclear Engineering, Tokyo Institute of Technology	1119923
11:20	Comparison of Gamma Dose Rate Calculations for PWR Spent Fuel Assemblies	Bo Feng	Argonne National Laboratory	1106285
11:40	SPENT FUEL CANISTER CRITICALITY CALCULATION IN GROUNDWATER PENETRATION ACCIDENT	si yuan wu	East China Institute of Technology, Ministry of Education	1092274

2-2. Deterministic Transport Theory (Hiei)

10:20	Iterative Properties of the Integral Transport Matrix Method for the DD Scheme in 2D Cartesian Geometry	Dmitriy Anistratov	North Carolina State University	1106264
10:40	CONVERGENCE ACCELERATION OF MULTI-COLLISION TECHNIQUE FOR LINEAR TRANSPORT PROBLEMS	Paolo Picca	Department of System and Industrial Engineering, the University of Arizona	1085479
11:00	Neutron Leakage Treatment in Reactor Physics: Consequences on SFR Characteristics Prediction	Gerald Rimpault	CEA	1104627
11:20	REVISIT BOUNDARY CONDITIONS FOR THE SELF-ADJOINT ANGULAR FLUX FORMULATION	Yaqi Wang	Idaho National Laboratory	1102705
11:40	Accuracy Preserving Surrogate for Neutron Transport Caculations	Congjian Wang	North Carolina State University	1127699

13. Radiation Applications and Nuclear Safeguards (Atago)

10:20	DESIGN OF LONG NEUTRON COUNTER FOR INTENSIFIED D-T NEUTRON SOURCE	Yanan Li	Institute of Nuclear Energy Safety Technology, CAS · FDS Team	1126839
10:40	Utilizing Simulated Rossi-Alpha Distributions to Develop New Methods of Characterizing Spent Nuclear Fuel	Alexis C Kaplan	LANL / University of Michigan	1104803
11:00	ANTINEUTRINO EMISSION FROM FUELS WITH HIGH PROLIFERATION RESISTANCE	Tomooki SHIBA	SUBATECH Laboratory	1123449
11:20	Validation of the Implicit Correlation Method in MCPNX-PoliMi using Plutonium Cross-Correlation Measurements	Matthew James Marcath	University of Michigan, Ann Arbor	1105483
11:40	A UNIQUE TUNGSTEN-BASED TAGGING APPROACH FOR MAINTAINING OF CONTINUITY OF KNOWLEDGE OF NUCLEAR WASTE COPPER CANISTERS	Dina Chernikova	Chalmers University of Technology	1127570

9-1. Transient and Safety Analysis (Cosmos)

10:20	DEVELOPMENT OF NEUTRON KINETIC CODE FOR MOLTEN SALT REACTOR	youqi zheng	Xi'an Jiaotong University	1084871
10:40	STUDY OF NEUTRON PROPAGATION IN MULTIGROUP TRANSPORT BY SPACE ASYMPTOTIC METHODS	Julio C.L. Fernandes	Politecnico di Torino	1102531
11:00	Sensitivity Analysis and Performance of the Adiabatic, Theta, and Multigrid Amplitude Function Kinetics Methods in 2D MOC Neutron	Samuel Shaner	Massachusetts Institute of Technology	1105035
11:20	Prompt Behavior of Generalized-Eigenvalue Point Kinetics Models	Brian C Kiedrowski	Los Alamos National Laboratory	1107334
11:40	Computations of heterogeneous dilution transients using CFX and HEMERA V1	Ludovic Maas	IRSN	1094627

September 30 (Tue.) 13:30-15:40

1-5. Reactor Analysis Method (Mizuho_A)

13:30	AUTOMATIC CONSTRUCTION OF A SIMPLIFIED BURN-UP CHAIN MODEL BY THE SINGULAR VALUE DECOMPOSITION	Takanori Kajihara	Hokkaido University	1105724
13:50	Generation of simplified burnup chain using contribution matrix of nuclide production	Ryota Katano	Nagoya University	1101303
14:10	IMPORTANT FISSION PRODUCT NUCLIDES IDENTIFICATION METHOD FOR SIMPLIFIED BURNUP CHAIN CONSTRUCTION	Go CHIBA	Hokkaido university	1068216
14:30	Application of Backtracking Algorithm to Depletion Calculations	Mingyu Wu	Wu Mingyu	1065760
14:50	Modeling the Cross Section of Gadolinia Pins in the Depletion for Pin-by-Pin Core Calculations	Yasuhiro Kodama	Nuclear Fuel Industries, Ltd.	1105480
15:10	THE OPTIMIZED ALGORITHM FOR THE MICROSCOPIC DEPLETION MODEL IN THE COCAGNE CORE CODE A 2-LEVEL CORE PARTITIONING APPROACH	Matthieu Raju Guillo	EDF R&D	1127599

4-1. Verification, Validation and Uncertainty Analysis (Mizuho_B)

13:30	Validation and benchmarking of calculation methods for photon and neutron transport at cask configurations	Luc Schlomer	WTI Wissenschaftlich-Technische Ingenieurberatung GmbH	1086809
13:50	RECENT ADVANCES IN THE V&V OF THE NEW FRENCH CEA APOLLO3R NEUTRON TRANSPORT CODE : BENCHMARKS ANALYSIS OF THE FLUX SOLVERS	Jean-Marc Palau	CEA	1086829
14:10	DECART CODE VERIFICATIONS BY NUMERICAL BENCHMARK CALCULATIONS OF HTTR	Chag Joon Jeong	Korea Atomic Energy Research Institute	1101312
14:30	DEVELOPMENT AND VERIFICATION OF THREE-DIMENSIONAL HEX-Z BURNUP SENSITIVITY SOLVER BASED ON GENERALIZED PERTURBATION THEORY	Kenji Yokoyama	Japan Atomic Energy Agency	1106037
14:50	Validation of HELIOS for ATR Core Follow Analyses	Samuel E Bays	Idaho National Laboratory	1081829

15:10	PERSENT: NEED OF A DETERMINISTIC CODE FOR SENSITIVITY ANALYSIS IN 3D GEOMETRY AND TRANSPORT THEORY	GERARDO ALIBERTI	Argonne National Laboratory	1100850

2-3. Deterministic Transport Theory (Hiei)

13:30	Axial Transport Solvers for the 2D/1D Scheme in MPACT	Shane Gray Stimpson	University of Michigan	1105193
13:50	Development of Legendre Expansion of Angular Flux Method for 3D MOC Calculation	Yuki KATO	Nagoya University	1106157
14:10	BENCHMARK ON DETERMINISTIC TIME-DEPENDENT TRANSPORT CALCULATIONS WITHOUT SPATIAL HOMOGENISATION	Victor F. Boyarinov	NRC Kurchatov Institute	1102516
14:30	Coarse-grained parallelism for full-core transport calculations	Roland Lenain	CEA France	1127828
14:50	PARALLEL PERFORMANCE RESULTS FOR THE OPENMOC METHOD OF CHARACTERISTICS CODE ON MULTI-CORE	William Boyd	MIT	1104798
15:10	MAKING MORE PRECISE THE SURFACE PSEUDOSOURCES METHOD FOR RBMK CLUSTER CELLS	Nikolay V. Sultanov	NRC "Kurchatov institute"	1103956

3-2. Monte Carlo Methods (Atago)

13:30	A MONTE CARLO METHOD FOR PROMPT AND DELAYED ALPHA EIGENVALUE CALCULATIONS	Andrea Zoia	CEA/Saclay	1099127
13:50	GEOMETRY NAVIGATION ACCELERATION BASED ON AUTOMATIC NEIGHBOR SEARCH AND ORIENTED BOUNDING BOX IN MONTE CARLO SIMULATION	Zhenping Chen	Institute of Nuclear Energy Safety Technology, CAS · FDS Team	1104233
14:10	Continuous-Energy Monte Carlo Methods for Calculating Generalized Response Sensitivities using TSUNAMI-3D	Christopher Michael Perfetti	Oak Ridge National Laboratory	1127880
14:30	Enhancements in Continuous-Energy Monte Carlo Capabilities in	Bradley Thomas Rearden	Oak Ridge National Laboratory	1142583
14:50	Leakage-corrected fast reactor assembly calculation with Monte-Carlo code TRIPOLI4 and its validation methodology	li cai	CEA,DEN, DER/SPRC	1102576
15:10	Impact of Nearest Neighbor Distribution of Fuel Particle on Neutronics Characteristics in Statistical Geometry Model	Takahiro Koide	Nagoya University	1101295

12-1. Fuel Cycle and Actinide Management (Cosmos)

13:30	DEVELOPMENT OF A FAST BURNUP CALCULATION TOOL FOR SIMULATION OF FUEL CYCLE WITH MINOR ACTINIDE	Mate Szieberth	Budapest University of Technology and Economics, Institute of Nuclear	1127550
13:50	Variations in activity, toxicity and decay heat of nuclear waste of various fuel cycles	Nicolas Emile Stauff	Argonne National Laboratory	1104106
14:10	EFFECT OF HETEROGENEITY IN PLUTONIUM RECYCLING IN STEADY STATE PWR	Marc Ernoult	Institut de Physique Nucleaire	1105805
14:30	EVALUATION METHOD OF EQUIVALENCE FACTORS FOR MOX FUEL AND NON-LINEAR “Equivalent Pu-239” FORMULA	Mikio TOKASHIKI	Nuclear Fuel Industries, Ltd.	1105974
14:50	Development of a Fuel Performance Code for Thorium-Plutonium Fuel	Klara Linnea Insulander Bjork	Thor Energy and Chalmers University of Technology	1127573
15:10	Two-Stage Fuel Cycles with Accelerator-Driven Systems	Florent Heidet	Argonne National Laboratory	1106238

September 30 (Tue.) 15:55-18:05

1-6. Reactor Analysis Method (Mizuho_A)

15:55	MODELLING OF SHUTDOWN COOLING REACTIVITY EFFECTS WITH SIMULATE	Tamer Bahadir	Studsvik Scandpower, Inc.	1106193
16:15	IMPLEMENTATION AND VERIFICATION OF THE SDM IN THE TITAN 3-D Sn TRANSPORT CODE	Nathan John Roskoff	Virginia Tech	1106648
16:35	TRANSPORT CORE SOLVER VALIDATION FOR THE ASTRID CONCEPTUAL DESIGN STUDY WITH APOLLO3R	Jean-Francois Vidal	CEA Cadarache	1101764
16:55	METHODOLOGY ASSESSMENT FOR THE EVALUATION OF THE COOLANT VOID WORTH IN SODIUM FAST REACTORS WITH A LOW VOID EFFECT CORE DESIGN	SARA BORTOT	Paul Scherrer Institut (PSI)	1099616
17:15	Results of verification of computer codes used for analysis of BN-1200 reactor core neutronics	Elena Marova	OKBM	1104321
17:35	APOLLO3R based method for 3D warped cores calculations ; Application to flowering tests of Phenix	Cyril Patricot	CEA	1106232

4-2. Verification, Validation and Uncertainty Analysis (Mizuho_B)

15:55	BENCHMARK CALCULATION WITH MOSRA-SRAC FOR BURNUP OF A BWR FUEL ASSEMBLY	Kensuke Kojima	Japan Atomic Energy Agency	1126354
16:15	VERIFICATION OF THE COCAGNE CORE CODE USING CLUSTER DEPLETION CALCULATIONS	Fabrice Hoareau	EDF R&D	1094607
16:35	LWR Fuel Reactivity Depletion Verification Using 2D Full Core MOC and Flux Map Data	Geoffrey A Gunow	Massachusetts Institute of Technology	1106258
16:55	CASMO-4E and CASMO-5 Analysis of the Isotopic Compositions of the LWR-PROTEUS Phase II Burnt PWR UO2 Fuel Samples	Peter Grimm	Paul Scherrer Institute ()PSI)	1104066
17:15	EXPERIMENTAL VALIDATION OF DECAY HEAT CALCULATIONS WITH VESTA 2.1	Wim Haeck	IRSN	1102701
17:35	DEVELOPMENT AND VALIDATION OF AD HOC ORIGEN-ARP LIBRARIES FOR VERY HIGH BURNUP UO2 PWR FUEL WITH	Stefano Caruso	National Cooperative for the Disposal of Radioactive Waste (NAGRA)	1095732

2-4. Deterministic Transport Theory (Hiei)

15:55	Phase Space Bases for Response Matrix Methods	Jeremy A Roberts	Kansas State University	1107355
16:15	The Drift Diffusion Limit of Thermal Neutrons: Theoretical and Numerical Results	Pablo A Vaquer	Texas A&M University	1126267
16:35	Adequacies of Different Convergence Accuracy Measures in Full-Core Nodal Flux Computations	Rene van Geemert	AREVA GmbH	1137347

16:55	FLEXIBLE SEMI-ANALYTICAL CALCULATION METHOD OF ESCAPE PROBABILITY	Tetsuo Matsumura	CRIEPI	1094291
17:15	CORRECTED DIAMOND DIFFERENCE METHOD FOR COUPLING FROM THE METHOD OF CHARACTERISTICS TO DISCRETE	Mitchell T.H. Young	University of Michigan	1106304
17:35	ENERGY MULTIGROUP SPECTRAL GREEN´S FUNCTION CONSTANT NODAL METHOD FOR FIXED-SOURCE Sn PROBLEMS IN X,Y-GEOMETRY	Ricardo C. Barros	Universidade do Estado do Rio de Janeiro	1121396

3-3. Monte Carlo Methods (Atago)

15:55	LARGE-SCALE MONTE CARLO CALCULATIONS WITH THERMAL-HYDRAULIC FEEDBACK	Aleksandar Stoyanov Ivanov	Karlsruher Institut fur Technologie	1127876
16:15	SODIUM VOID REACTIVITY EFFECT ANALYSIS USING THE NEWLY DEVELOPED EXACT PERTURBATION THEORY IN MONTE-CARLO CODE TRIPOLI-4	Guillaume Truchet	CEA, DEN, DER/SPRC/LEPh, Cadarache	1104427
16:35	Monte Carlo Perturbation Analysis on Isothermal Temperature Reactivity Coefficient of Light-Water Moderated and Reflected Critical	Byoung Kyu Jeon	Seoul National University	1105946
16:55	Monte Carlo and Thermal-Hydraulic Coupling via PVMEXEC	Daniel F. Gill	Bechtel Marine Propulsion Corporation	1142507
17:15	Perturbation Based Monte Carlo Criticality Search in Density, Enrichment and Concentration	Zeguang LI	Tsinghua University	1104699
17:35	Monte Carlo Perturbation Method for Geometrical Uncertainty Analysis using McCARD	Ho Jin Park	Korea Atomic Energy Research Institute	1105031

12-2. Fuel Cycle and Actinide Management (Cosmos)

15:55	Thorium-Fueled Breed-and-Burn Fuel Cycle	Florent Heidet	Argonne National Laboratory	1106234
16:15	Fuel Cycle Analysis of a Self-Sustaining Light Water Cooled Reactor with 232Th/233U Fuel and Impact of 233U (n,γ) Cross Section	Nicholas R Brown	Brookhaven National Laboratory	1104784
16:35	PRODUCTION OF 232U FROM IRRADIATION OF STANDARD AND THORIUM-BASED FUELS IN PWR REACTORS	Baptiste Leniau	Subatech	1105816
16:55	FUEL CYCLE SCHEME DESIGN AND EVALUATION FOR THORIUM-URANIUM BREEDING RECYCLE IN CANDU REACTORS	Bo YANG	Shanghai Nuclear Engineering Research and Design Institute	1087546
17:15	An Inventory Analysis of Thermal-Spectrum Thorium-Fueled Molten Salt Reactor Concepts	Jeffrey J. Powers	Oak Ridge National Laboratory	1126252
17:35	Agent-Based Dynamic Resource Exchange in Cyclus	Matthew J. Gidden	UW - Madison	1104469

October 1 (Wed.) 8:00-10:05

1-7. Reactor Analysis Method (Mizuho_A)

8:00	High order source approximaion for the EFEN method	Yunzhao Li	School of Nuclear Science and Technology, Xi'an Jiaotong University	1106187
8:20	EXTENSION OF LINEAR SOURCE MOC METHODOLOGY TO ANISOTROPIC SCATTERING IN CASMO5	Rodolfo M. Ferrer	Studsvik Scandpower, Inc.	1099700
8:40	FINITE DIFFERENCE EQUATIONS FOR NEUTRON FLUX AND IMPORTANCE DISTRIBUTION IN A HETEROGENEOUS REACTOR WITHOUT HOMOGENIZATION AND DIFFUSION APPROXIMATION	Aleksandr V Elshin	Alexandrov Research Institute of Techology, Rosatom	1104610
9:00	EFFICIENT SUBSPACE CONSTRUCTION FOR REDUCED ORDER MODELING IN REACTOR ANALYSIS	Bassam A. Khuwaileh	Department of Nuclear Engineering North Carolina State University, Raleigh, NC,	1126237
9:20	Variational Acceleration of Fission Source Iteration for Subcritical Source-Driven Systems	Bilge Ozgener	Istanbul Technical University	1124804
9:40	An Incident Flux Coupling Calculation Study for Nodal Method and Monte Carlo Method	Xinzhe Wang	China Institute of Atomic Energy	1104763

4-3. Verification, Validation and Uncertainty Analysis (Mizuho_B)

8:00	Confidence interval estimation by bootstrap method for uncertainty quantification using random sampling method	Tomohiro Endo	Nagoya University	1084668
8:20	Uncertainty quantification of neutronics characteristics using Latin Hypercube Samplig method	Kuniharu Kinoshita	Nagoya University	1101192
8:40	Uncertainty Quantification of BWR Core Characteristics using Latin Hypercube Sampling Method	Akio Yamamoto	Nagoya University	1090063
9:00	Applicability of the cross section adjustment method based on random sampling technique for burnup calculation	Tomoaki Watanabe	Nagoya University	1098930
9:20	NUSS-RF: Stochastic Sampling-Based Tool for Nuclear Data Sensitivity and Uncertainty Quantification	Ting Zhu	Paul Scherrer Institut	1102712
9:40	MOCABA: A General Monte Carlo-Bayes Procedure for Improved Predictions of Integral Functions of Nuclear Data	Axel Hoefer	AREVA GmbH Offenbach, Germany	1104105

SS3. Hybrid Particle Transport Methods for Solving Complex Problems in Real-Time (Hiei)

8:00	DISCRETIZED MESH TOOLS AND RELATED TREATMENT FOR HYBRID TRANSPORT APPLICATION WITH 3D DISCRETE ORDINATES AND MONTE CARLO	Kevin L Manalo	Georgia Institute of Technology	1106684
8:20	A NOVEL HYBRID WEIGHTING SCHEME FOR MULTI-GROUP CROSS SECTION COLLAPSING	Ce Yi	Georgia Institute of Technology	1106722
8:40	Development of An Iterative Lattice-Core Coupling Method Based on MICROX-2 Cross Section Libraries	Jason Hou	The Pennsylvania State University	1104276
9:00	SOLUTION OF A STYLIZED EUROPEAN PRESSURIZED REACTOR (EPR) BENCHMARK PROBLEM USING THE COARSE MESH RADIATION TRANSPORT METHOD (COMET)	Daniel Lago	Georgia Institute of Technology	1106263
9:20	COMPUTATIONAL EFFICIENCY AND ACCURACY OF THE FISSION COLLISION SEPARATION METHOD IN 3D HTTR BENCHMARK PROBLEMS	Dingkang Zhang	Georgia Institute of Technology	1106314

9:40	Use of the Fission Matrix Method for Solution of the Eigenvalue Problem in a Spent Fuel Pool	William J Walters	Virginia Tech	1107387

3-4. Monte Carlo Methods (Atago)

8:00	XSBENCH - THE DEVELOPMENT AND VERIFICATION OF A PERFORMANCE ABSTRACTION FOR MONTE CARLO REACTOR ANALYSIS	John Robert Tramm	Argonne National Laboratory	1106774
8:20	Development of Neutron Current Connection Method for Whole Core Analysis Based on Monte Carlo Method	Naoyuki Nakadozono	Hitachi Research Laboratory, Hitachi, Ltd.	1126312
8:40	Analysis of Select BEAVRS PWR Benchmark Cycle 1 Results Using MC21 and OpenMC	Daniel J. Kelly	Bechtel Marine Propulsion Corporation	1138691
9:00	Monte Carlo Neutronics Analysis of Sodium-cooled fast Reactor Benchmark with OTF Temperature and Burnup Treatment	Nicole Simone Guilliard	University Stuttgart - IKE	1127863
9:20	VERIFICATION OF COUPLED 3D FUEL CYCLE ANALYSIS WITH STABLE MONTE CARLO BASED CODE, BGCORE, AGAINST THE NODAL DIFFUSION DYN3D CODE	Dan Kotlyar	Ben Gurion University	1126263
9:40	Domain Decomposition and Terabyte Tallies with the OpenMC Monte Carlo Neutron Transport Code	Nicholas E Horelik	Massachusetts Institute of Technology	1106210

9-2. Transient and Safety Analysis (Cosmos)

8:00	RECRITICALITY RISK IN PWR SPENT FUEI POOLS	Guillaume Grandjean	University of Fukui Research Institute of Nuclear Engineering	1081521
8:20	DEMONSTRATION OF FULLY COUPLED SIMPLIFIED EXTENDED STATION BLACK-OUT ACCIDENT SIMULATION WITH RELAP-7	Haihua Zhao	Idaho National Laboratory	1100842
8:40	CODE SCALING APPLICABILITY TO A COLD LEG SBLOCA SCE-NARIO IN A NUCLEAR POWER PLANT	Andrea Querol	Universitat Politecnica de Valencia	1106251
9:00	TRANSIENT SIMULATION OF GAS BUBBLE IN A MEDIUM SIZED LEAD COOLED FAST REACTOR	Carl Fredrik Hellesen	Uppsala University	1106269
9:20	Power Ramp Transient in a Sodium-Cooled Fast Reactor used for Minor Actinides Transmutation	Sara Perez-Martin	Karlsruhe Institute of Technology	1127822
9:40	STEP TOWARDS INTEGRAL VALIDATION OF ENERGETIC RE-CRITICALITY PREDICTION FOR SODIUM COOLED FAST REACTOR	Tatiana Ivanova	Institut de Radioprotection et de Surete Nucleaire (IRSN)	1126197

October 1 (Wed.) 10:20-12:00

1-8. Reactor Analysis Method (Mizuho_A)

10:20	Modernization Enhancements in SCALE 6.2	Bradley T. Rearden	Oak Ridge National Laboratory	1153741
10:40	SALOME-CORE platform: uses for EDF R&D neutronic studies	Hadrien Leroyer	EDF R&D	1086739
11:00	A STEADY-STATE CORE ANALYSIS CODE FOR THE MODELING OF ACCELERATOR-DRIVEN SUBCRITICAL REACTORS	Shengcheng Zhou	Xi'an Jiaotong University	1085680
11:20	Good Practice in Development of Advanced Assembly/Core Calculation Methods and Implementations of AEGIS/SCOPE2	Masahiro Tatsumi	Nuclear Engineering Ltd.	1105705
11:40	VALIDATION OF LANCR01/AETNA01 BWR CODE PACKAGE AGAINST FUBILA MOX EXPERIMENTS AND FUKUSHIMA DAIICHI NUCLEAR POWER PLANT UNIT 3 MOX CORE	Tatsuya Iwamoto	Core Design Group, Global Nuclear Fuel-Japan	1100982

4-4. Verification, Validation and Uncertainty Analysis (Mizuho_B)

10:20	Pinpower Uncertainty Quantification of LWR-PROTEUS Phase III	Mathieu Hursin	Paul Scherrer Institut	1099077
10:40	QUANTIFICATION OF CODE, LIBRARY AND CROSS-SECTION UNCERTAINTY EFFECTS ON THE VOID REACTIVITY COEFFICIENT OF A BWR UO2 ASSEMBLY	Olivier Leray	Paul Scherrer Institut	1100815
11:00	SENSITIVITY AND UNCERTAINTY ANALYSIS OF BURNUP REACTIVITY FOR AN ACCELERATOR-DRIVEN SYSTEM	Hiroki Iwamoto	Japan Atomic Energy Agency	1104912
11:20	SENSITIVITY/UNCERTAINTY ANALYSIS FOR BWR CONFIGURATIONS OF EXERCISE I-2 OF UAM BENCHMARK	Nuria Garcia-Herranz	Universidad Poltecnica de Madrid	1106021
11:40	IMPACT OF THE FISSION YIELD NUCLEAR DATA UNCERTAINTIES IN THE PIN-CELL BURN-UP OECD/NEA UAM BENCHMARK	Oscar Cabellos	Universidad Politecnica de Madrid	1105867

8-1. Reactor Operation and Safety (Hiei)

10:20	INTERPRETATION OF EXPERIMENTAL DATA ON SAFETY PARAMETERS FROM STANDPOINT OF MATHEMATICAL NUCLEAR REACTOR THEORY	Alexey A. Mekhryushev	National Reseach Nuclear University "MEPhI"	1104432
10:40	Effects of cross sections libraries parameters on the OECD/NEA Oskarshamn-2 Benchmark solution	Paolo Balestra	ENEA	1126097
11:00	EVALUATION OF OPERATIONAL EXPERIENCES AND REACTOR PHYSICS TESTS OF MOX LOADED BWR CORES	Yusuke Kuroda	TEPCO SYSTEMS CORPORATON	1086934
11:20	Coupling effects in large reactor cores: the impact of heavy and conventional reflectors on power distribution perturbations	Antonio Sargeni	IRSN	1099602
11:40	IMPROVE THE ACCURACY OF THE POWER DISTRIBUTION RECONSTRUCTION USING POWER DISTRIBUTIONS OF DIFFERENT STATUS AS THE FUNDAMENTAL HARMONIC	Kai FAN	Tsinghua University	1093593

6-1. Reactor Physics Experiments (Atago)

10:20	Static Modal Analysis of the Current-to-Flux Subcriticality Monitor for Accelerator-Driven Systems	Wim Uyttenhove	SCK-CEN, Belgian Nuclear Research Centre	1104770
10:40	12 YEARS OF FRANCO-JAPANESE INTERNATIONAL PROGRAMS IN EOLE FOR THE VALIDATION OF 100%MOX RECYCLING IN LWRS	Patrick BLAISE	DEN,CAD, Department of Reactor Studies, Experimental Physics Section - SPEX / DEN,CAD, Department of Nuclear Technology, - STRI	1104068

11:00	Application of the best representativity method to a future PWR fuel assembly calculation using four critical experiments of different facilities	Takuya UMANO	Toshiba Corporation Power Systems Company	1104323
11:20	CHARACTERIZATION OF IRRADIATION FIELDS IN THE EXPERIMENTAL FAST REACTOR JOYO FOR FUEL AND MATERIAL	Shigetaka Maeda	Japan Atomic Energy Agency	1098227
11:40	Reaction Rate, Fission Product Yield, and Rossi-Alpha Measurements Using a HEU Metal, Copper Reflected Critical Assembly	Rene G. Sanchez	Los Alamos National Laboratory	1105192

9-3. Transient and Safety Analysis (Cosmos)

10:20	SIMMER-III MODELING OF GAS COOLED FAST REACTOR	Xue-Nong Chen	Karlsruhe Institute of Technology (KIT), Institute for Nuclear and Energy Technologies (IKET)	1106163
10:40	VALIDATION OF THE SUBCHANNEL CODE CTF AGAINST THE BENCHMARK DATA OF THE OECD/NEA PSBT	Patricio Hidalga	Universitat Politecnica de Valencia	1106259
11:00	VALIDATION OF CASMO5 / SIMULATE-3K USING THE SPECIAL POWER EXCURSION TEST REACTOR III E-CORE: COLD START-UP, HOT START-UP, HOT STANDBY AND FULL POWER	Gerardo M. Grandi	Studsvik Scandpower, Inc.	1099703
11:20	VALIDATION OF THE NODAL KINETICS CODE SYSTEM GALAXY/COSMO-K USING THE SPERT-III E-CORE EXPERIMENTS	Kazuya Yamaji	Mitsubishi Heavy Industries, Ltd.	1119885
11:40	Propagation of Nuclear Data Uncertainty for a Control Rod Ejection Accident using the Total Monte-Carlo Method	Dirceu F. da Cruz	Nuclear Research and Consultancy Group NRG	1102512

October 1 (Wed.) 13:30-15:40

1-9. Reactor Analysis Method (Mizuho_A)

13:30	Resonant Upscattering Effects on U238 Absorption Rates	CLAUDE MOUNIER	Commissariat a l'energie atomique et aux energies alternatives	1106006
13:50	The up-scattering treatment in the fine-structure self-shielding method in APOLLO3	Li LEI-MAO	CEA	1107332
14:10	Target Motion Sampling Temperature Treatment Technique with Track-length Estimators in OpenMC - Preliminary Results	Tuomas Viitanen	VTT Technical Research Centre of Finland	1106078
14:30	Problem-Dependent Doppler Broadening of Continuous-Energy Cross Sections in the KENO Monte Carlo Computer Code	Shane William Daniel Hart	The University of Tennessee	1101879
14:50	VERIFICATION OF DOUBLY-HETEROGENEOUS SELF-SHIELDING METHOD BASED ON EQUIVALENCE THEORY	Sooyoung Choi	Ulsan National Institute of Science and Technology	1106138
15:10	Quantification of Resonance Interference Effect for Multi-Group Effective Cross-Section in Lattice Physics Calculation	Hiroki Koike	Mitsubishi Heavy Industries, Ltd.	1105375

4-5. Verification, Validation and Uncertainty Analysis (Mizuho_B)

13:30	Deterministic approach of the decay heat uncertainty due to JEFF-3.1.1 nuclear data uncertainties with the CYRUS tool and the DARWIN2.3 depletion code	Vanessa Vallet	CEA/DEN	1104036
13:50	Bias and Uncertainty Assessment of Pressurized Water Reactor Fuel Isotopics	Ryan Nathaniel Bratton	The Pennsylvania State University	1127866
14:10	INVENTORY UNCERTAINTY QUANTIFICATION AND PROPAGATION USING TENDL COVARIANCE DATA IN FISPACT-II	jean-christophe cyrille Sublet	United Kingdom Atomic Energy Authority	1103900
14:30	Validation of CASMO5 spent fuel isotopics with decay and fission yield uncertainties	Joshua Hykes	Studsvik Scandpower, Inc.	1104131
14:50	UNCERTAINTY PROPAGATION AND SENSITIVITY ANALYSIS IN THE ALEPH MONTE CARLO BURNUP CODE: APPLICATIONS TO FISSION PULSE DECAY HEAT CALCULATIONS	Luca Fiorito	SCKCEN	1106036
15:10	UNCERTAINTY QUANTIFICATION OF SPENT FUEL NUCLIDE COMPOSITIONS DUE TO CROSS SECTIONS, DECAY CONSTANTS AND FISSION YIELDS	Olivier Leray	Paul Scherrer Institut	1104088

7-1. Reactor Concepts and Designs (Hiei)

13:30	CONCEPTUAL STUDY OF A LONG-LIFE PGSFR	Donny Hartanto	KAIST	1104420
13:50	CORE DESIGN STUDIES ON THE FAST REACTOR WITH FLEXIBLE BREEDING RATIO	Yunlong Xiao	Xi an Jiaotong University	1092268
14:10	ENHANCED FEEDBACK EFFECTS IN SODIUM COOLED FAST REACTORS USING MODERATING MATERIAL - THE EFFECT OF THE PLUTONIUM CONTENT IN THE FUEL	Bruno Merk	Helmholtz-Zentrum Dresden-Rossendorf e.V.	1096317
14:30	ADVANCED SODIUM COOLED REACTOR CORES HAVING THORIUM BLANKETS FOR EFFECTIVE BURNING OF	Wu Seung YOU	Kyung Hee Univsersity	1093377
14:50	PHISICS IMPROVEMENTS AND COMPARATIVE STUDY WITH ERANOS 2.2 ON THE GEN-IV LEAD-COOLED FAST REACTOR CONCEPT, ALFRED	Francesco Lodi	University of Bologna	1104422
15:10	Velocity characteristic and stability of wave solutions for a candle reactor with thermal feedback	Volodymyr Khotyayintsev	Department of Physics, T. Shevchenko National University of Kyiv	1106054

3-5. Monte Carlo Methods (Atago)

13:30	Effects of Stochastic Noise on a Three-Dimensional Monte Carlo Depletion Analysis of the H.B. Robinson Reactor	Scott J Spychala	Bettis Atomic Power Laboratory	1142466
13:50	EFFICIENT ESTIMATION OF ADJOINT-WEIGHTED KINETICS PARAMETERS IN THE MONTE CARLO WIELANDT CALCULATIONS	Sung Hoon Choi	Seoul National University	1105965
14:10	HIGHER-ORDER CHEBYSHEV RATIONAL APPROXIMATION METHOD (CRAM)	Maria Pusa	VTT Technical Research Centre of Finland	1119422
14:30	Efficiency and Accuracy Evaulation of the Windowed Multipole Direct Doppler Broadening Method	Colin Josey	Massachusetts Institute of Technology	1126651
14:50	DIFFUSION THEORY MONTE CARLO METHOD WITH TRANSPORT CORRECTIONS	Argala Srivastava	BHABHA ATOMIC RESEARCH CENTRE, MUMBAI, INDIA	1105538

15:10	GENERATION OF ONE-GROUP SELF SHIELDED CROSS SECTIONS WITH MULTI-GROUP APPROACH FOR MONTE CARLO BURNUP CODES	Dan Kotlyar	Ben Gurion University	1084485

15. Research Related to Fukushima Accident (Cosmos)

13:30	AREVA Dismantling and Decommisioning experience and Fuel Debris removal approach for Fukushima Dai-ichi	Konrad Schauer	AREVA	2000001
13:50	RE-CRITICALITY POTENTIAL AT FUKUSHIMI DAI ICHI UNIT 4 FOLLOWING THE MARCH 2011 EARTHQUAKE AND TSUNAMI	Alan H. Wells	Electric Power Research Institute (EPRI)	1106299
14:10	Why a criticality excursion was possible in the Fukushima spent fuel	Antonio SARGENI	IRSN	1085679
14:30	CRITICAL EXPERIMENTS FOR FUEL DEBRIS USING MODIFIED	Kazuhiko IZAWA	Japan Atomic Energy Agency	1127170
14:50	CONCEPT OF CAPTURE CREDIT BASED ON NEUTRON INDUCED GAMMA RAY SPECTROSCOPY	Yasushi Nauchi	Central Research Institute of Electric Power Industry	1105423
15:10	A methodology for fast and accurate decay heat calculations for in-pool used fuel assemblies developed at AREVA La Hague reprocessing	Agnes Launay	AREVA NC Recycling	1125870

October 2 (Thu.) 8:00-10:05

1-10. Reactor Analysis Method (Mizuho_A)

8:00	An Asymptotic Homogenized SP2 Approximation to the Boltzmann Equation. I. Derivation	Thomas G. Saller	University of Michigan	1106112
8:20	An Asymptotic Homogenized SP2 Approximation to the Boltzmann Equation. II. Discontinuity Factors and Numerical Testing	Thomas G. Saller	University of Michigan	1106177
8:40	GENERALIZED AND STANDARD MULTIGROUP NEUTRON DIFFUSION EQUATION EIGENVALUE PROBLEM WITH THE FINITE VOLUME METHOD	Alvaro Bernal	Universitat Politecnica de Valencia	1106303
9:00	A Generalized Multigroup Method Based on Finite Elements	Andrew T Till	Texas A&M University	1127687
9:20	AN ASYMPTOTIC, HOMOGENIZED, ANISOTROPIC, MULTIGROUP DIFFUSION APPROXIMATION TO THE NEUTRON TRANSPORT EQUATION	Travis John Trahan	Los Alamos National Laboratory	1105474
9:40	ACCURACY OF THE LINEAR DISCONTINUOUS GALERKIN METHOD FOR REACTOR ANALYSIS WITH RESOLVED FUEL PINS	Carolyn N McGraw	Texas A&M University	1127761

4-6. Verification, Validation and Uncertainty Analysis (Mizuho_B)

8:00	SENSITIVITY AND UNCERTAINTY CALCULATIONS METHODS OF NEUTRONICS PARAMETERS IN PWR CORES PART I: THEORY AND SENSITIVITY CALCULATIONS	Toshikazu TAKEDA	Research Institute of Nuclear Engineering, University of Fukui	1100398
8:20	Constrained Quantities in Uncertainty Quantification: Ambiguity and Tips to Follow	Zoltan Perko	TU Delft, Department of Radiation Science and Technology, Nuclear Energy and Radiation Applications	1084579
8:40	ADJOINT-BASED SENSITIVITY AND UNCERTAINTY ANALYSIS OF LATTICE PHYSICS CALCULATIONS WITH CASMO-4	Maria Pusa	VTT Technical Research Centre of Finland	1106024
9:00	Nuclear data uncertainty propagation on power maps in large LWR	Alain Santamarina	CEA	1106197
9:20	Are Modeling Uncertainties Properly Considered in Neutronics Data Assimilation Analysis?	Paridhi Athe	North Carolina State University	1126688
9:40	Sensitivity Analysis via Reduced Order Adjoint Method	Youngsuk Bang	FNC Technology, Co., Ltd	1106183

7-2. Reactor Concepts and Designs (Hiei)

8:00	SmAHTR-CTC Neutronic Design	Dan Ilas	Oak Ridge National Laboratory	1154037
8:20	SUPPRESSION OF EXCESS REACTIVITY OF SMALL LONG-LIFE PRISMATIC HTGR WITH PASSIVE DECAY-HEAT REMOVAL	Odmaa Sambuu	Tokyo Institute of Technology	1105504
8:40	CONCEPTUAL DESIGN OF A SELF-SUSTANABLE PRESSURIZED WATER REACTOR WITH BOILING CHANNELS	Marat Margulis	Ben Gurion University	1091293
9:00	Feasibility Study of Application of BigT	HwanYeal Yu	KAIST	1105145
9:20	Axially Homogeneous Thorium Fuel Designs for Transuranic Burning in Reduced-moderation BWRs	Benjamin A. Lindley	University of Cambridge	1068763
9:40	Neutronic Analysis of A Micro Modular Reactor	Francesco Venneri	Ultra Safe Nuclear Corporation	1127098

6-2. Reactor Physics Experiments (Atago)

8:00	AN IMPROVED FEYNMAN-α CORRELATION ANALYSIS WITH A MOVING-BUNCHING TECHNIQUE	Ryohei Okuda	Interdisciplinary Graduate School of Science and Engineering, Kinki University	1102247
8:20	PULSED NEUTRON AND SOURCE JERK EXPERIMENTS FOR REACTIVITY ASSESSMENT IN DEEP SUBCRITICAL CONFIGURATION: A CASE STUDY WITHIN THE FRAMEWORK OF	Giorgia Mila	INFN - Section of Turin	1128133
8:40	AN ALTERNATIVE SOURCE JERK METHOD IMPLEMENTATION FOR THE SUBCRITICALITY ESTIMATION OF THE VENUS-F SUBCRITICAL CORE IN THE FREYA PROJECT	Anatoly Kochetkov	SCK-CEN, Belgium Nuclear Research Centre	1105864
9:00	Reactivity Measurement of the Lead Fast Subcritical VENUS-F Reactor using Beam Interruption Experiments	Thibault Matthieu Chevret	LPC Caen, ENSICAEN/Universite de Caen/CNRS-IN2P3	1087437
9:20	REACTIVITY MEASUREMENTS AT GUINEVERE FACILITY USING THE INTEGRAL kp METHOD	Sebastien Chabod	CNRS	1104440
9:40	ESTIMATION OF THE DELAYED NEUTRON FRACTION β_eff OF THE MAESTRO CORE IN MINERVE ZERO POWER REACTOR	Erez Gilad	Department of Nuclear Engineering, Ben-Gurion University of Negev	1090478

10-1. Nuclear Data (Cosmos)

8:00	COMAC: Nuclear Data Covariance Matrices Library for Reactor	Pascal Archier	CEA, DEN, DER, SPRC, LEPh	1077966
8:20	Model-based Generation of Neutron Induced Fission Yields up to 20 MeV by the GEF Code	Kilian P. Kern	Karlsruhe Institute of Technology	1106067

8:40	EXPERIMENTS TO IMPROVE UNCERTAINTY OF THE 1st DELAYED NEUTRON GROUP ABUNDANCE IN FAST FISSIONS OF U238 AND SENSITIVITY STUDIES OF THE RELATIVE PARAMETERS	Heejun Chung	University of Florida	1104110
9:00	OBSERVATION OF NEUTRON THERMALIZATION IN GRAPHITE USING THE SLOWING-DOWN-TIME TECHNQIUE	Ayman I. Hawari	North Carolina State University	1105487
9:20	MEASUREMENT OF NEUTRON CAPTURE CROSS SECTION OF 232TH IN THE LOW ENERGY REGION	Jun-ichi Hori	Research Reactor Institute, Kyoto University	1127786
9:40	New Revisions of Reactor Physics Standards	Dimitrios Cokinos	Brookhaven National Laboratory	1113347

October 2 (Thu.) 10:20-12:00

1-11. Reactor Analysis Method (Mizuho_A)

10:20	Revisiting the Clio perturbative approach for analyzing systems in fundamental mode conditions	Alain Hebert	Ecole Polytechnique de Montreal	1104443
10:40	A Generalization of Lambda-Mode Xenon Stability Analysis	Justin Michael Pounders	Bettis Atomic Power Laboratory	1127646
11:00	Preliminary Study of the Impact of Xe-135m on the PCR of CANDU	Jaeha Kim	KAIST	1105707
11:20	Verification of the spectral history correction method with fully coupled Monte-Carlo code BGCore	Yurii Bilodid	Helmholtz-Zentrum Dresden-Rossendorf	1106131
11:40	CORE NEUTRONICS METHODOLOGIES APPLIED TO THE MOX-LOADED KAIST 1A BENCHMARK: REFERENCE TO INDUSTRIAL CALCULATIONS	Ansar CALLOO	EDF R&D/SINETICS	1092925

4-7. Verification, Validation and Uncertainty Analysis (Mizuho_B)

10:20	Effective Physics-Based Uncertainty Quantification for ZrHx Thermal Neutron Scattering in TRIGA Reactors	Weixiong Zheng	Texas A&M University	1104568
10:40	Sensitivity and uncertainty analysis on reactor kinetic parameters using per-turbation theory	Cyrille Bouret	Commissariat Energie Atomique et aux Energies Alternatives	1126308
11:00	Selecting benchmarks for reactor calculations	Erwin Alhassan	Division of Applied Nuclear Physics, Department of Physics and Astronomy Uppsala University	1106307
11:20	DICE 2013: NEW CAPABILITES AND DATA	Ian P Hill	OECD/NEA	1105736
11:40	OECD NEA Benchmark Database of Spent Nuclear Fuel Isotopic Compositions for World Reactor Designs	Ian C Gauld	Oak Ridge National Laboratory	1106253

8-2. Reactor Operation and Safety (Hiei)

10:20	ADVANCED SURVEILLANCE OF RESISTANCE TEMPERATURE DETECTORS IN NUCLEAR POWER PLANTS.	Cristina Montalvo	Iberdrola Engineering & Construction	1099698
10:40	REFINED METHOD FOR SURVEILLANCE AND DIAGNOSTICS OF THE CORE BARREL VIBRATIONS OF THE RINGHALS PWRs	Imre Pazsit	Chalmers Univ. of Technology	1104708
11:00	ASSESSMENT OF FLOW INDUCED VIBRATION LIMITS IN PRELIMINARY I2S-LWR FUEL DESIGNS	Glenn E. Sjoden	Georgia Institute of Technology	1106266
11:20	BLOCKAGE INDEX FOR THE DETECTION OF FLOW BLOCKAGE IN A SUBASSEMBLY OF SODIUM-COOLED FAST REACTOR	Hae-Yong Jeong	Sejong University	1122850
11:40	SECOND GENERATION SHIELDING ASSEMBLIES - NEUTRON FLUX IMPACT ON REACTOR PRESSURE VESSEL AND CORE	Klaes-Hakan Bejmer	Vattenfall AB Sweden	1101789

6-3. Reactor Physics Experiments (Atago)

10:20	BNL Metal Fuel Lattice Experiments: Candidates for Reactor Physics Benchmark Evaluation	Nicholas R Brown	Brookhaven National Laboratory	1104785
10:40	Benchmark Evaluation of the Neutron Radiography (NRAD) Reactor Upgraded LEU-Fuel Core	John Darrell Bess	Idaho National Laboratory	1084758
11:00	CALIBAN AND GODIVA-IV MEASUREMENTS USING HELIUM-3 DETECTOR SYSTEMS	Jesson Hutchinson	LANL	1105154
11:20	POWER SPECTRAL ANALYSIS FOR A SUBCRITICAL REACTOR SYSTEM DRIVEN BY A PULSED SPALLATION NEUTRON SOURCE	Atsushi Sakon	Interdisciplinary Graduate School of Science and Engineering, Kinki University	1102582
11:40	Equivalencies of Open & Closed Loop Reactivity Measurements	Benjamin A Baker	Idaho State University	1130969

SS4. Advanced Geometry Processing in Deterministic and Monte Carlo Methods (Cosmos)

10:20	DEVELOPMENT OF A MULTI-GROUP SN TRANSPORT CALCULATION CODE WITH UNSTRUCTURED TETRAHEDRAL	SER GI HONG	Kyung Hee Univsersity	1093366
10:40	DEVELOPMENT OF AN UNSTRUCTURED MESH BASED GEOMETRY MODEL IN THE SERPENT 2 MONTE CARLO CODE	Jaakko Leppanen	VTT Technical Research Centre of Finland	1101586
11:00	A CAD BASED AUTOMATIC MODELING METHOD FOR PRIMITIVE SOLID BASED MONTE CARLO CALCULATION GEOMETRY	Dong Wang	Institute of Nuclear Energy Safety Technology, CAS・ FDS Team	1105335
11:20	Developments and Applications of the Geometry Conversion Tool McCad for Monte Carlo Particle Transport Simulation	Lei Lu	Karlsruhe Institute for Technology (KIT)	1106105
11:40	A MEMORY EFFICIENT ALGORITHM FOR CLASSIFYING UNIQUE REGIONS IN CONSTRUCTIVE SOLID GEOMETRIES	Derek M Lax	Massachusetts Institute of Technology	1106231

October 2 (Thu.) 13:30-15:40

1-12. Reactor Analysis Method (Mizuho_A)

13:30	THE ROLE OF THE EIGENVALUE SEPARATION IN REACTOR DYNAMICS AND NEUTRON NOISE THEORY	Imre Pazsit	Chalmers University of Technology	1106213
13:50	INVESTIGATION OF CONDITIONAL TRANSPORT UPDATE IN METHOD OF CHARACTERISTICS BASED COARSE MESH FINITE DIFFERENCE TRANSIENT CALCULATION	Yeon Sang Jung	Seoul National University	1106109
14:10	IMPLEMENTATION OF AN A PRIORI TIME STEP ESTIMATOR FOR THE MULTIGROUP NEUTRON DIFFUSION EQUATION IN ASYNCHRONOUSLY COUPLED RELAP5-3D	Michael Wayne Hackemack	Texas A&M University	1127681

14:30	DEVELOPMENT OF A THREE-DIMENSIONAL KINETICS CODE FOR COMMERCIAL-SCALE FBR FULL CORE ANALYSIS	Yoichiro Shimazu	University of Fukui Research Institute of Nuclear Engineering	1081523
14:50	USE OF ADJOINT FUNCTIONS FOR COMPARING MEASURED AND CALCULATED PARAMETERS IN THE SUBCRITICAL SYSTEMS	Sergey Shevchenko	SEC NRS	1091810
15:10	DEVELOPMENT OF THE NEUTRON SOURCE EVALUATION METHOD AND PREDICTOR OF SRM/SRNM COUNT RATE IN BWR	Masayuki Tojo	Global Nuclear Fuel-Japan	1097414

14. Education in Reactor Physics (Mizuho_B)

13:30	TBD	Kouji Hiraiwa	TOSHIBA Corporation	2000003
13:50	Past, Present and Future of MIT Reactor Physics	Benoit Forget	Massachusetts Institute of Technology	1105147
14:10	NEW PRACTICAL EXERCISES AT THE JSI TRIGA MARK II	Luka Snoj	Jozef Stefan Institute	1106257
14:30	Developing a course in nuclear reactor modelling and going from campus-based to web-based teaching	Christophe R. Demaziere	Chalmers University of Technology	1099675
14:50	Introduction to the status of Reactor physics Education in Tsinghua	ganglin yu	Tsinghua University	1106245
15:10	Reactor Physics Education at Seoul National University	Han Gyu Joo	Seoul National University	1127896

7-3. Reactor Concepts and Designs (Hiei)

13:30	IMPROVING THE CARBIDE LEU-NTR ROCKET PERFORMANCE	Paolo Francesco Venneri	Korea Advanced Institute of Science and Technology	1126817
13:50	Molybdenum-99 Production in the Oregon State TRIGA Reactor: Analysis of Multiple Smaller Core Designs Using a New LEU Target as	Andrew J Hummel	Oregon State University	1106761
14:10	Preliminary Design of the Delft Isotope Production Reactor (DIPR)	Jan Leen Kloosterman	Delft University of Technology	1104016
14:30	A STUDY OF SAFETY CORE DESIGN ON BEAM TRANSIENT FOR ACCELERATOR DRIVEN SYSTEM	Naoto Aizawa	Tohoku University	1105750
14:50	NEW INVERTED HYDRIDE FUEL DESIGN CONCEPT FOR PRESSURE TUBE TYPE SUPER CRITICAL WATER REACTORS	Liangzhi Cao	Xi'an Jiaotong University	1080898
15:10	Preliminary safety analysis of a Thorium Breeder Pebble Bed Reactor	Frank Jozef Wols	Delft University of Technology	1069253

6-4. Reactor Physics Experiments (Atago)

13:30	Overview of the 2014 Edition of the International Handbook of Evaluated Reactor Physics Benchmark Experiments (IRPhEP Handbook)	John Darrell Bess	Idaho National Laboratory	1102703
13:50	Analysis of Tungsten Gray Rods Critical Experiments Using PARAGON with Ultra-Fine Energy Mesh Methodology	Mohamed Ouisloumen	Westinghouse Electric Company LLC	1101855
14:10	Monte Carlo assessment of spatial and energy effects in the VENUS-F subcritical configurations and application for reactivity determination.	Vicente Becares	CIEMAT	1127823
14:30	VALIDATION OF ORIGEN2 COUPLED WITH JENDL-4.0 BASE LI-BRARIES FOR ISOTOPIC COMPOSITIONS OF IRRADIATED LIGHT WATER REACTOR FUELS	Toru Yamamoto	Nuclear Regulation Authority	1103841
14:50	THE AMMON EXPERIMENT IN EOLE FACILITY: A CHALLENGING PROGRAM DEDICATED TO THE EXPERIMENTAL VALIDATION OF JHR NEUTRONIC AND PHOTONIC CALCULATION TOOLS	Jacques DI SALVO	CEA, DEN, DER/SPEx, Cadarache	1099684
15:10	Monte Carlo analysis of reactivity effect measurements in the AMMON experimental program dedicated to JHR neutron studies	Claire Vaglio-Gaudard	CEA - Commissariat a l'Energie Atomique et aux Energies Alternatives	1101619

10-2. Nuclear Data (Cosmos)

13:30	Impact of the interpolation mode on the secondary particles for shielding and criticality benchmarks with TRIPOLI-4 Monte Carlo Code	cedric Jouanne	CEA Saclay	1104046
13:50	Impact of the differences in nuclear data on estimated k-effective of SFR	Nicolas Emile Stauff	Argonne National Laboratory	1104107
14:10	NUCLEAR DATA SENSITIVITY ANALYSIS FOR ISOTOPIC GENERATION USING JENDL-4.0, ENDF/B-VII.1 AND JEFF-3.1.1	Yosuke Kawamoto	Nuclear Reactor Engineering Laboratory	1105258
14:30	VALIDATION OF A POINTWISE ENERGY NEUTRON CROSS SECTION LIBRARY GENERATED BY RXSP-BETA2.0 USING ENDFB-	Jiankai YU	Department of Engineering Physics, Tsinghua University	1102714
14:50	CRITICALITY ANALYSES OF URANIUM ZIRCONIUM CARBON NITRIDE LEU FUEL WITH ENDF/B-VII.1, JEFF-3.1.2, AND JENDL-4 NUCLEAR DATA FILES	Alberto Talamo	Argonne National Laboratory	1068258
15:10	FEEDBACK ON 239PU AND 240PU NUCLEAR DATA AND ASSOCIATED COVARIANCES THROUGH THE CERES INTEGRAL	Pierre LECONTE	CEA Cadarache	1104041