

Poster Presentations

The name appeared in the following program is the first author of each paper.

Notifications on presenter is recorded by the organization committee and they are used to check the registration status of each paper.

October 1 (Wed.) 16:15-18:15 (Mizuho_C, Mizuho_D)

Note on the poster session:

-Size of poster board: 90cm(width) by 200cm(height)

-Preparation of poster display : Oct.1 (Wed.), 14:00 - 16:00

-Removal of poster : Oct.1 (Wed.), 18:15 - 19:00

-Best poster award will be selected through the evaluation of technical paper and poster presentation

Track1 Reactor Analysis Method

1089476	A Parametric Study and Comparison of BWR Fuel Depletion Calculations using CASMO-4, MCNPX, and SCALE/TRITON	Ching-Sheng Lin	Institute of Nuclear Engineering and Science
1092007	NEUTRON NOISE INDUCED BY FLUCTUATIONS OF THE BORIC ACID CONTENT IN PRESSURIZED WATER REACTORS	Hessam Malmir	Department of Energy Engineering, Sharif University of Technology
1093007	APPLICATION OF WESTINGHOUSE NEXUS/ANC9 CROSS-SECTION MODEL FOR PWR ACCIDENT ANALYSES	Baocheng Zhang	Westinghouse Electric Company
1102611	NUMERICAL DISPERSION AND DISSIPATION ANALYSIS OF NODAL EXPANSION METHOD	Xiaofeng Zhou	Institute of Nuclear and New Energy Technology, Tsinghua University
1102622	The Integration of Control Rod Calculation and VSOP	Jiong Guo	Institute of Nuclear and New Energy Technology, Tsinghua University
1103763	REFLECTOR MODELLING WITH MULTI-GROUP NODAL EQUIVALENCE THEORY FOR THE SAFARI-1 RESEARCH REACTOR	Suzanne Anel Groenewald	South African Nuclear Energy Corporation (Necsa)
1104477	RELATIONSHIP BETWEEN COMPUTED ANSI/ANS-5.1 AND ORIGEN-S DECAY HEAT POWERS FOR BWR LOCA SAFETY ANALYSIS	Ren-Tai Chiang	Energy Engineering Service
1104519	A New Method To Measuring The α Eigenvalue of A subcritical reactor system	yun bai	IAPCM
1105008	PROGRESS TOWARDS AN ACCURATE LATTICE-HOMOGENIZATION TECHNIQUE FOR PRESSURE-TUBE SUPERCRITICAL WATER COOLED REACTOR NEUTRONIC CALCULATIONS	Eleodor Nichita	University of Ontario Institute of Technology
1105869	PROPOSAL OF SUBCRITICAL PWR CORE BENCHMARK PROBLEMS	Takanori Kitada	Osaka university
1105924	A NEW MONTE CARLO-DETERMINISTIC TWO-STEP METHOD FOR FAST REACTOR DIFFUSION ANALYSIS	Woong Heo	KAIST
1106049	Explicit Transverse Leakage Treatment Using an Analytic Basis Function Expansion	Steven A Thompson	The Pennsylvania State University
1106127	Method for calculation capturing reactions contribution to total energy release in nuclear reactors.	Rynat Bekirovich Bahdanovich	The Belarusian State University
1106156	A PRELIMINARY ANALYSIS OF THE ACCURACY OF HOMOGENIZED 2D CROSS SECTION IN 3D NODAL CALCULATIONS FOR BWRs	Andrew M. Ward	University of Michigan
1106178	Deterministic Lattice Code Development at UNIST	Chidong Kong	Ulsan National Institute of Science and Technology
1106220	Implications of mesh refinement in lattice physics on BWR core analysis and nuclear design	Petri Forslund Guimaraes	Westinghouse Electric Sweden AB
1106272	GENERATING MULTIGROUP DATA STOCHASTICALLY FOR A HIGHLY HETEROGENEOUS VHTR PROBLEM	Kevin John Connolly	Georgia Institute of Technology
1106273	REFERENCE SOLUTION FOR CORE PHYSICS	Radim Vocka	UJV Rez, a.s.
1106738	Development of A Cross Section Library Applicable to Various Reactor Types	Changho Lee	Argonne National Laboratory
1119837	Monte Carlo Analysis of Doppler Reactivity Coefficient for UO2 Pin Cell Geometry	Yasunobu Nagaya	Japan Atomic Energy Agency
1125254	Assessment of the depletion capability in MPACT	Ang Zhu	University of Michigan
1125817	MONTE CARLO MODELLING OF VR-1 REACTOR CORE	Tomas Bily	Czech Technical University in Prague, FNSPE, Department of Nuclear Reactors
1126966	ASSESSMENT OF THE 2D/1D IMPLEMENTATION IN MPACT	Benjamin S Collins	University of Michigan
1127524	Coupled Neutronics and Thermal-Hydraulic Solution of a Full-Core PWR using VERA-CS	Scott Palmtag	Core Physics Inc.
1127710	ASSESSMENT OF THE WIMS9A/PARCS CODE SYSTEM FOR POWER DENSITY CALCULATIONS OF THE WESTINGHOUSE AP1000 REACTOR	Mohamed A. Elsawi	Khalifa University of Science, Technology and Research
1127773	Feasibility of Nodal Equivalence Theory Using Functionalized Discontinuity Factors	Woosong Kim	KAIST
1127819	THE MULTIGROUP NEUTRONIC MODEL OF NUSTAR'S 3D CORE CODE EGRET	Shaohong Zhang	Shanghai NuStar Nuclear Power Technology Co., Ltd.
1128004	SIMULATION OF WATTS BAR INITIAL STARTUP TESTS WITH CONTINUOUS ENERGY MONTE CARLO METHODS	Andrew T. Godfrey	Oak Ridge National Laboratory

Track2 Deterministic Transport Theory

1039082	The application and performance of ACMFD acceleration in 2D/3D full core MOC transport fuse method	Zhiyong Li	China Nuclear Power Technology Research Institute Shanghai Branch
1083797	A COUPLING METHOD OF SUBGROUP AND WAVELET EXPANSION FOR THE RESONANCE PARAMETER CALCULATION	hongchun Wu	Xi'an Jiaotong university
1104084	Verification of Ray Effect Elimination Module in the Transport Code ARES	MengTeng CHEN	North China Electric Power University

Track3 Monte Carlo Methods

1084078	3-D MONTE CARLO NEUTRON-PHOTON TRANSPORT CODE JMCT AND ITS ALGORITHMS	Li Deng	Institute of Applied Physics and Computational Mathematics
1099694	NEUTRON CHANNELS SHIELD DESIGN ANALYSES OF KIPT NEUTRON SOURCE FACILITY	Zhaopeng Zhong	Argonne National Laboratory
1101864	Development of a new convergence criterion for Monte Carlo coupled simulation with thermal-hydraulics feedback	XU WU	University of Illinois at Urbana-Champaign
1102407	JCOGIN: A PARALLEL PROGRAMMING INFRASTRUCTURE FOR MONTE CARLO PARTICLE TRANSPORT	Baoyin Zhang	Institute of Applied Physics and Computational Mathematics
1104378	Void transit time calculations by neutron noise of propagating perturbation using complex-valued weight Monte Carlo	Toshihiro Yamamoto	Kyoto University
1105897	Simulating Fast Transients with Fuel Behavior Feedback using the Serpent-2 Monte Carlo code	Ville Valtavirta	VTT Technical Research Centre of Finland
1105995	STATUS OF MONTE CARLO CODE DEVELOPMENT AT UNIST	Hyunsuk Lee	UNIST
1106262	Criticality Benchmarking of ANET Monte Carlo Code	Thalia Xenofontos	NCSR 'Demokritos', Institute of Nuclear and Radiological Sciences & Technology, Energy & Safety
1123217	Effective diffusion Homogenization of Cross Sections with the Monte Carlo method	Dusan Calic	ZEL-EN d.o.o. / Institute Jozef Stefan
1228305	ESTIMATING LOCAL IN- AND EX-CORE RESPONSES WITHIN MONTE CARLO SOURCE ITERATION EIGENVALUE CALCULATIONS	Kenneth William Burn	ENEA

Track4 Verification, Validation and Uncertainty Analysis

1067489	Review of Neutronic Assessments applied to small reactor core physics	Laurent Chabert	AREVA TA
1084893	A new neutronics analysis code system for fast reactors	Toshikazu Takeda	Research Institute of Nuclear Engineering, University of Fukui
1100864	Uncertainty Analysis of Delayed Neutron Fissile Material Assay Using a Genetic Algorithm	Ryan P. Kelley	University of Florida
1102809	IR APPROXIMATION FOR CALCULATING SENSITIVITY AND UNCERTAINTY OF PWR CELLS BY TAKING ACCOUNT OF SELF-SHIELDING EFFECT	Basma FOAD	University of Fukui
1102810	UNCERTAINTY QUANTIFICATION OF REACTOR KINETICS PARAMETERS USING JENDL-4.0 COVARIANCE DATA	Go Chiba	Hokkaido university
1103703	Survey on Effect of Crystal Texture of Beryllium on Total Cross-section to Improve Neutronic Evaluation in JMTR	Noriyuki Takemoto	Japan Atomic Energy Agency
1103924	UNCERTAINTY AND SENSITIVITY ANALYSIS FOR AN OECD/NEA HTGR BENCHMARK WITH XSUSA	Alexander Aures	Gesellschaft für Anlagen- und Reaktorsicherheit (GRS) mbH
1104067	First Verification and Validation Steps of MENDEL Release 1.0 Code System	Sebastien Lahaye	CEA/DEN/DANS/DM2S/SERMA/LLPR
1104316	Application of the GRS method for estimation of uncertainties of LMFBR type reactor physics parameters with taking into account macroscopic experiments	Anton Peregudov	Institute for Physics and Power Engineering
1104670	BENCHMARKING OF PHOTON AND COUPLED NEUTRON AND PHOTON PROCESS OF SUPERMC 2.0	Jing Song	Institute of Nuclear Energy Safety Technology, CAS - FDS Team
1104787	In Depth Uncertainty Estimation of the Neutron Computational Tools	Gerald Rimpault	CEA
1105709	UPDATE OF THE PSI CRITICALITY SAFETY EVALUATION METHODOLOGY USING MCNPX2.7 AND ENDF/B-VII.1	Marco Pecchia	Paul Scherrer Institut (PSI)
1105825	VALIDATION OF TWO MONTE CARLO CODES FOR LWR BURNUP CALCULATIONS	Damar Wicaksono	Ecole polytechnique federale de Lausanne
1106101	BENCHMARKING OF DeCART2D AGAINST CRITICAL EXPERIMENTS	Kyung-Hoon Lee	Korea Atomic Energy Research Institute
1106250	AUTOMATED REACTOR RECORDS EVALUATION FRAMEWORK	Jonatan Hejzlar	UJV Rez, a.s.
1109432	Monte Carlo Based Equilibrium Cycle Analysis of One-Dimensional Breed and Burn Benchmark Problem	Zhiwen Xu	TerraPower LLC
1126078	Validation of the Monte Carlo Code RMC with a PWR MOXUOX Core Benchmark	Xiaotong SHANG	Department of Engineering Physics Tsinghua University, REAL team
1126219	INVERSE UNCERTAINTY QUANTIFICATION FOR NUCLEAR DATA ASSESSMENT	Bassam A. Khuwaileh	Department of Nuclear Engineering North Carolina State University, Raleigh, NC, USA
1127444	ON HOW SENSITIVE THE CROSS-SECTION SENSITIVITY CALCULATIONS ARE TO PN ORDER APPROXIMATIONS	Ivan Alexander Kodeli	Jozef Stefan Institute
1137713	VALIDATION OF NUSTAR'S PWR CORE ANALYSIS SYSTEM	Shaohong Zhang	Shanghai NuStar Nuclear Power Technology Co., Ltd.

Track6 Reactor Physics Experiments

1085573	DETERMINATION OF THE ^{58}Ni (n,p) ^{58}Co REACTION CROSS SECTION FOR BOTH GROUND AND ISOMERIC STATES	Zdena Lahodova	Research Centre Rez Ltd.
1086633	Investigation on Subcriticality Measurement Using Inherent Neutron Source in Nuclear Fuel	Takeshi Shiozawa	Nagoya University
1094641	BENCHMARK CALCULATIONS OF SODIUM FAST CRITICAL EXPERIMENTS	Elena Mitenkova	Nuclear Safety Institute of Russian Academy of Sciences
1104329	CRITICAL EXPERIMENTS FOR BWR FUEL ASSEMBLIES WITH THE CLUSTER OF GADOLINIA RODS	Kenichi YOSHIOKA	TOSHIBA Corporation
1104817	THE CALCULATION AND MEASUREMENT OF FAST NEUTRON REFLECTION IN THE VVER-1000 MOCK-UP MODEL PLACED IN THE LR-0 REACTOR	Michal Kostal	Research Center Rez
1105052	DEVELOPMENT OF REACTIVITY METER WITH NOVELTY NEUTRON SOURCE INTENSITY EVALUATION MODEL FOR BWR APPLICATION	Shoichi Tashiro	Global Nuclear Fuel-Japan Co., Ltd.
1106265	Boron Carbide Neutron Screen for GRR-1 Neutron Spectrum Tailoring	Nefeli Chrysanthopoulou	NCSR "Demokritos", INRaSTES, Research Reactor Lab
1106850	RESEARCH AND DEVELOPMENT ACTIVITIES FOR TRANSMUTATION PHYSICS EXPERIMENTAL FACILITY IN J-PARC	Takanori Sugawara	JAEA

1108237	Analysis of TCA criticality, β_{eff} and β_{eff}/l using CASMO-4 and CASMO-5	shigeaki aoki	MNF
1126731	Measurement of Subcriticality using Delayed Neutron Source Combined with Pulsed Neutron Accelerator	Tsuyoshi Misawa	Kyoto University Research Reactor Institute
1127355	ROSSI- α PARAMETER MEASUREMENT OF DALAT NUCLEAR REACTOR BY ANALYSIS OF CROSS POWER SPECTRAL DENSITY OBTAINED FROM 2 ION CHAMBERS	Tuan Minh Nguyen	Nuclear Research Institute, Dalat
1127688	ANALYSIS OF INTEGRAL EXPERIMENT FOR THORIUM FUEL CYCLE AT KYOTO UNIVERSITY CRITICAL ASSEMBLY	Yoshiyuki Takahashi	Research Reactor Institute, Kyoto University
1127907	STUDIES OF POTENTIAL FOR CONVERSION OF THE IDAHO NATIONAL LABORATORY TREAT TRANSIENT TEST REACTOR TO LOW-ENRICHMENT FUEL	Dimitrios C. Kontogeorgakos	Idaho National Laboratory

Track7 Reactor Concepts and Designs

1068764	Axially Heterogeneous Thorium Fuel Designs for Transuranic Burning in Reduced-moderation BWRs	Benjamin A. Lindley	University of Cambridge
1090085	Method Development and Reactor Physics Data Evaluation for Improving Prediction Accuracy of Fast Reactors' Minor Actinides Transmutation Performance	Toshikazu Takeda	Research Institute of Nuclear Engineering, University of Fukui
1094719	A long life sodium cooled fast reactor concept with radial shuffling	Zhipeng Li	Xi'an Jiaotong University
1099566	Critical Boron Concentration Reduction Method in a Core Design	Chang Joo Hah	KEPCO International Nuclear Graduate School (KINGS)
1104423	BigT - A New Burnable Absorber Concept for PWR	Mohd-Syukri Yahya	KAIST
1104448	PRELIMINARY DESIGN OF A SPHERICAL BREED/BURN REACTOR	Elias Yammir Garcia-Cervantes	National Autonomous University of Mexico
1104495	OPTIMIZATION OF ULTRA-LONG CYCLE FAST REACTOR CORE	Taewoo Tak	Ulsan National Institute of Science and Technology
1106189	THE MAIN CHARACTERISTICS OF THE EVOLUTION PROJECT VVER-S WITH SPECTRUM SHIFT REGULATION	Pavel S Teplov	Center Nuclear Technology NRC "Kurchatov Institute"
1127032	ALTERNATIVE CORES FOR A MULTIPURPOSE EXPERIMENTAL SODIUM-COOLED FAST REACTOR WITH U-ZR FUEL	Tae-Yang Noh	Kyung Hee University
1127409	PRELIMINARY EVALUATION OF COOLANT VOID REACTIVITY OF A RE-ENTRANT CHANNEL PRESSURE-TUBE SUPERCRITICAL WATER COOLED REACTOR	Peter Schwanke	UOIT

Track8 Reactor Operation and Safety

1092230	The Development of SOPHORA -PHYSOR2014	Wenhui Li	China Nuclear Power Technology Research Institute
1105990	DEVELOPMENT OF RISK MONITOR RISKANGEL FOR RISK-INFORMED APPLICATIONS IN NUCLEAR POWER PLANTS	Fang Wang	Institute of Nuclear Energy Safety Technology, CAS · FDS Team
1127006	STEADY-STATE SUBCHANNEL ANALYSIS OF PARTIALLY BLOCKED COOLANT CHANNELS IN A POOL-TYPE TRIGA REACTOR	Jean C Ragusa	Texas A&M University

Track9 Transient and Safety Analysis

1084717	IMPROVEMENT OF SPACE-TIME KINETICS CAPABILITY IN THE SNATCH SOLVER AND COMPARISON TO KIN3D/PARTISN RESULTS	Maxime Guyot	CEA Cadarache
1094483	Different methods to model the MSLB without primary cooling pumps using HEMERA V1 system codes	Benoit NORMAND	IRSN
1101709	SOME RESULTS OF STUDYING OF SPATIAL KINETICS IN FAST REACTORS	Irina Panova	Nuclear Safety Institute of Russian Academy of Sciences
1103760	ATUCHA 2 OBLIQUELY INSERTED CONTROL RODS RELAP5-3D/NESTLE MODEL	Raul Gonzalez Gonzalez	San Piero a Grado Nuclear Research Group (GRNSPG)
1104619	The SIMMER/PARTISN Capability for Severe Accident Analyses	Marco Marchetti	Karlsruhe Institute of Technology
1105969	Development of a high-fidelity Monte Carlo thermal-hydraulics coupled code system Serpent/SUBCHANFLOW - First results	Miriam Daeubler	Karlsruhe Institute of Technology

Track10 Nuclear Data

1068671	ANALYSIS OF RADIOACTIVITY RATIOS OF FISSION PRODUCT NUCLIDES DEPOSITED TO SOIL IN FUKUSHIMA DAI-ICHI NUCLEAR POWER PLANT ACCIDENT	Go Chiba	Hokkaido university
1081806	EXPERIMENTAL UNCERTAINTY ESTIMATION IN PROFIL AND PROFIL-2 SAMPLE IRRADIATION EXPERIMENTS IN PHENIX	Edwin Privas	CEA Cadarache
1105105	UPDATED MULTI-GROUP CROSS SECTIONS OF MINOR ACTINIDES WITH IMPROVED RESONANCE TREATMENT	Muhammad Sohail	Kyung Hee University
1106244	PARAMETERIZED REPRESENTATION OF MACROSCOPIC CROSS SECTION FOR PWR REACTOR CONSIDERING WITH 12 BURNABLE ABSORBER FUEL RODS IN THE FUEL ELEMENT	Joao Claudio Batista Fiel	Military Institute of Engineering

Track11 Research Reactors and Spallation Sources

1087763	JULES HOROWITZ REACTOR.FRANCE EXPERIMENTAL LOOP DEVELOPMENT ACCORDING OPTIMIZED IRRADIATION PROCESS.	Stephane GAILLOT	CEA Cadarache
1106121	Preliminary Neutronic Design for the Conceptual Fluid Granular Spallation Target	Jinyang Li	Institute of Modern Physics, Chinese Academy of Sciences
1126142	Accumulation of tritium in beryllium slab under neutron irradiation	Sandybek Kunakov	Physico-Technical faculty, Al-Farabi Kazakh National University
1127738	A PRELIMINARY STUDY OF AN IMPROVED AREA METHOD, ADAPTED TO SHORT TIME TRANSIENTS IN SUB-CRITICAL SYSTEMS	Paolo Saracco	INFN
1128078	PRELIMINARY NEUTRONICS ANALYSIS OF A SPALLATION TARGET FOR TRANSMUTATION	Bin Wu	Institute of Nuclear Energy Safety Technology, CAS · FDS Team
1128332	PRELIMINARY OPTIMIZATION ANALYSIS OF THE RADIATION SHIELDING OF THE CHINA LEAD-BASED RESEARCH REACTOR	Qi Yang	Institute of Nuclear Energy Safety Technology, CAS · FDS Team

1233696	Preliminary Analysis of Radioactive Source Term for Normal Operation of China Lead-Alloy Cooled Research Reactor	Tongqiang Dang	Institute of Nuclear Energy Safety Technology, CAS · FDS Team
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Track12 Fuel Cycle and Actinide Management

1092834	FUSION HYBRIDS FOR GENERATION OF ADVANCED (231Pa+232U+233U+234U)-FUEL IN CLOSED (U-Pu-Th)-FUEL CYCLE	Gennady Genrikhovich Kulikov	National Research Nuclear University Moscow Engineering Physics Institute
1100837	233U FUEL PRODUCTION AND 30-YEAR UTILIZATION WITHOUT REPROCESSING AND REFUELLING USING HEAVY WATER COOLANT	alberto talamo	Argonne National Laboratory
1101727	UNCERTAINTY ANALYSIS FOR FUEL FLUX CALCULATIONS OF FAST REACTORS WITH EXTERNAL FUEL CYCLE	Evgeny Seleznev	Nuclear Safety Institute of Russian Academy of Sciences
1105292	STUDY ON TRANSMUTATION AND STORAGE OF LLFP USING A HIGH-TEMPERATURE GAS-COOLED REACTOR	Kazuki Kora	Kyushu University
1105802	CORE LIBRARY FOR ADVANCED SCENARIO SIMULATION, C.L.A.S.S. : PRINCIPLE & APPLICATION	Baptiste MOUGINOT	CNRS
1106324	Fuel Composition Generation Techniques of Nuclear Fuel Cycle Simulators	Robert Ryan Flanagan	University of Texas at Austin
1106382	Core Burnup Calculation of Uranium Rock-like Oxide Fuel PWR for Spent Fuel Composition Estimation	Hiroshi Akie	Japan Atomic Energy Agency

Track14 Education in Reactor Physics

1085651	MULTI-COLLISION THEORY FOR EDUCATED PEDESTRIANS	Paolo Picca	Department of System and Industrial Engineering, the University of Arizona
1073641	Virtual Labs on unique experimental equipment	Ivan S Saldikov	NRNU MEPHI
1104802	PINSPEC: A MONTE CARLO CODE FOR PIN CELL SPECTRAL CALCULATIONS FOR EDUCATIONAL APPLICATIONS	William Boyd	MIT
1127873	UNIQUE APPROACHES IN EMPHASIZING THE ROLE OF REACTOR LABORATORIES AND FACILITIES FOR TRAINING AND EDUCATION OF FUTURE NUCLEAR ENGINEERS 'WITHOUT THE BORDERS'	Tatjana Jevremovic	The University of Utah

Track15 Research Related to Fukushima Accident

1102869	Spatial Correlation Modeling of Macroscopic Cross Section with Weierstrass Function	Taro Ueki	Japan Atomic Energy Agency
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SpecialSession2 Reactor Physics and Criticality Safety Activities in OECD/NEA Working Party

1109848	EFFECTS OF NUCLEAR DATA LIBRARY AND ULTRA-FINE GROUP CALCULATION FOR LARGE SIZE SODIUM-COOLED FAST REACTOR OECD BENCHMARKS	Teruhiko KUGO	Japan Atomic Energy Agency
1121153	EVALUATION OF OECD/NEA/WPRS BENCHMARK ON MEDIUM SIZE METALLIC CORE SFR BY DETERMINISTIC CODE SYSTEM: MARBLE AND MONTE CARLO CODE: MVP	Mari Marianne Uematsu	JAEA
1195260	Quantifying the Effect of Undersampling in Monte Carlo Simulations using SCALE	Chris Perfetti	Oak Ridge National Laboratory

SpecialSession5 Multiscale, Multiphysics Approaches in Nuclear Science and Engineering Applications

1102894	Research on SCWR Core Characteristics Utilizing Pin-Wise Neutronics Thermal-Hydraulic Coupling Method	Qian HONG	Shanghai Nuclear Engineering Design & Research Institute
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SpecialSession6 Nuclear Criticality Safety of Fuel Debris

1087714	POST-ACCIDENT DEFUELING PROCEDURE AND ITS CRITICALITY SAFETY EVALUATION OF THE FUKUSHIMA-DAIICHI NUCLEAR POWER PLANTS	Naoyuki Takaki	TOKYO CITY UNIVERSITY
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